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on EURATOM Research and Training
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NUCLEAR SITE INTEGRATED CHARACTERIZATION FOR RADIOACTIVE WASTE MINIMIZATION: THE INSIDER PROJECT

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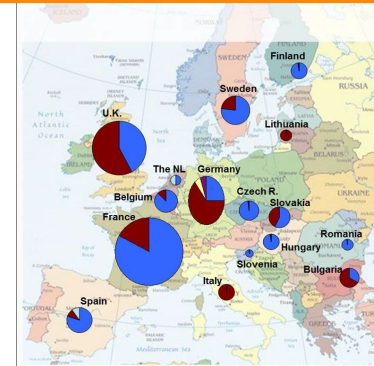


CONTENT

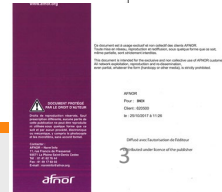
- ❑ Context and objectives of the INSIDER project
- ❑ Methodology
- ❑ Developments and implementation
- ❑ Preliminary benchmark results
- ❑ Perspectives and conclusions

CONTEXT

- **A global technical, societal, environmental and economic challenge for the 21st century**
 - **By 2050**, more than the half of today's 400 GW nuclear capacity around the world is scheduled to be shut down for decommissioning
 - Nuclear materials represent **a wide variety of matrices and contaminants**
- **An accurate fit for purpose radiological and chemical characterisation of facilities and sites** is required for dismantling and classification of contaminated materials.
 - Physical, radiological and non radiological characterisation prior to dismantling is a key element for all D&D projects (OECD, NEA, IAEA):
 - Scenario definition
 - Cost estimation
 - Radioactive waste production and categorisation
- **Smart applications and waste management routes** must be available to minimise the amount of radioactive waste and related potential hazard.
 - Need for reliable data to explore **different sustainable management routes** for contaminated materials: reuse, recycle...



[Sources: IAEA PRIS]



INSIDER project



Improved Nuclear Site characterisation for waste minimisation in D&D operations under constrained EnviRonment

❑ A EU-funded Horizon 2020 project

❑ “Research and innovation on the **overall management of radioactive waste other than geological disposal**”

❑ “Management of **non-standard waste** including D&D waste”

❑ **4-year project:** launched in **June 2017**

❑ What **INSIDER** will achieve

❑ To develop and validate **a new and improved integrated characterisation methodology and strategy** during nuclear decommissioning and dismantling operations (D&D) of nuclear power plants, post accidental land remediation or nuclear facilities **under constrained environments**.



Results will be validated through 3 case studies

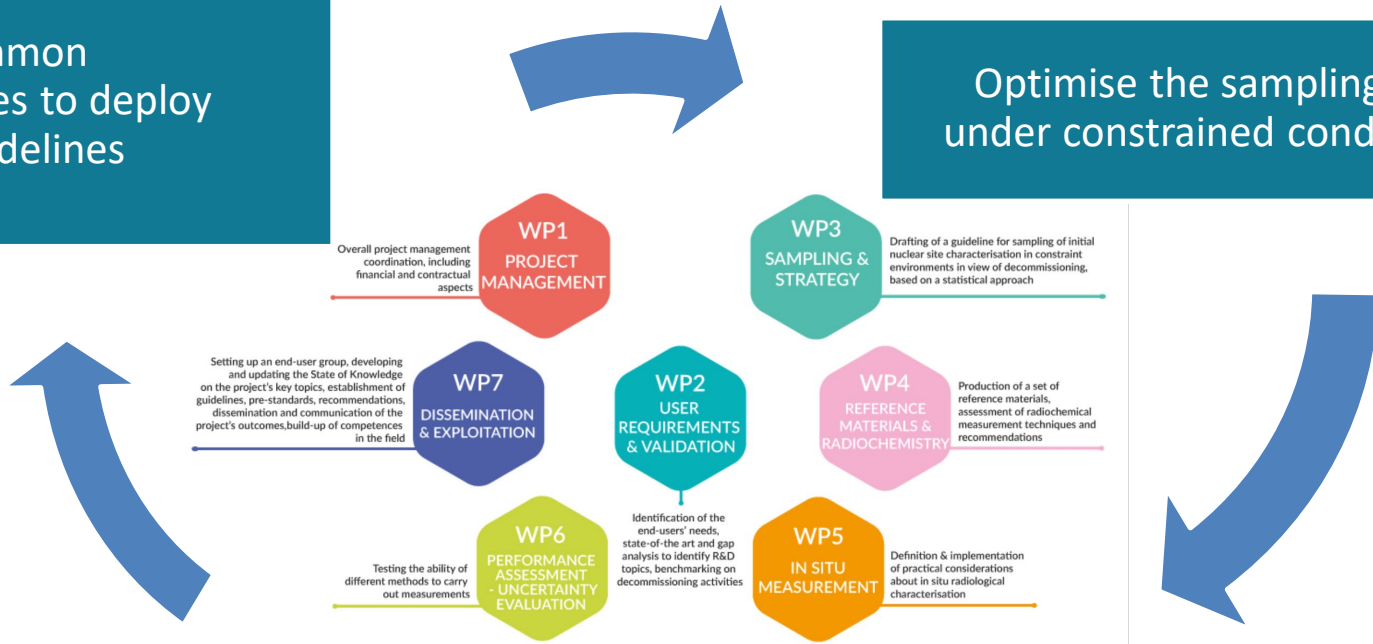


IAEA	EUG
IRSN	
ANDRA	
ENRESA	
SOGIN	
NDF	
Kraftanlagen Heidelberg	
KAERI	
ORANO	
IRE	
ENGIE	

Key objectives- Project organisation

Establish common methodologies to deploy reference guidelines

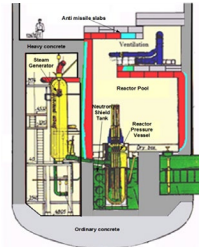
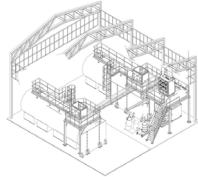
Optimise the sampling strategy under constrained conditions



Coupling sampling/measurement:
Performance assesement of available measurement techniques

Three case studies

Apply the methodologies to real worksites under decommissioning



1

Decommissioning of a back/end fuel cycle and/or research facility - Ispra (JRC)

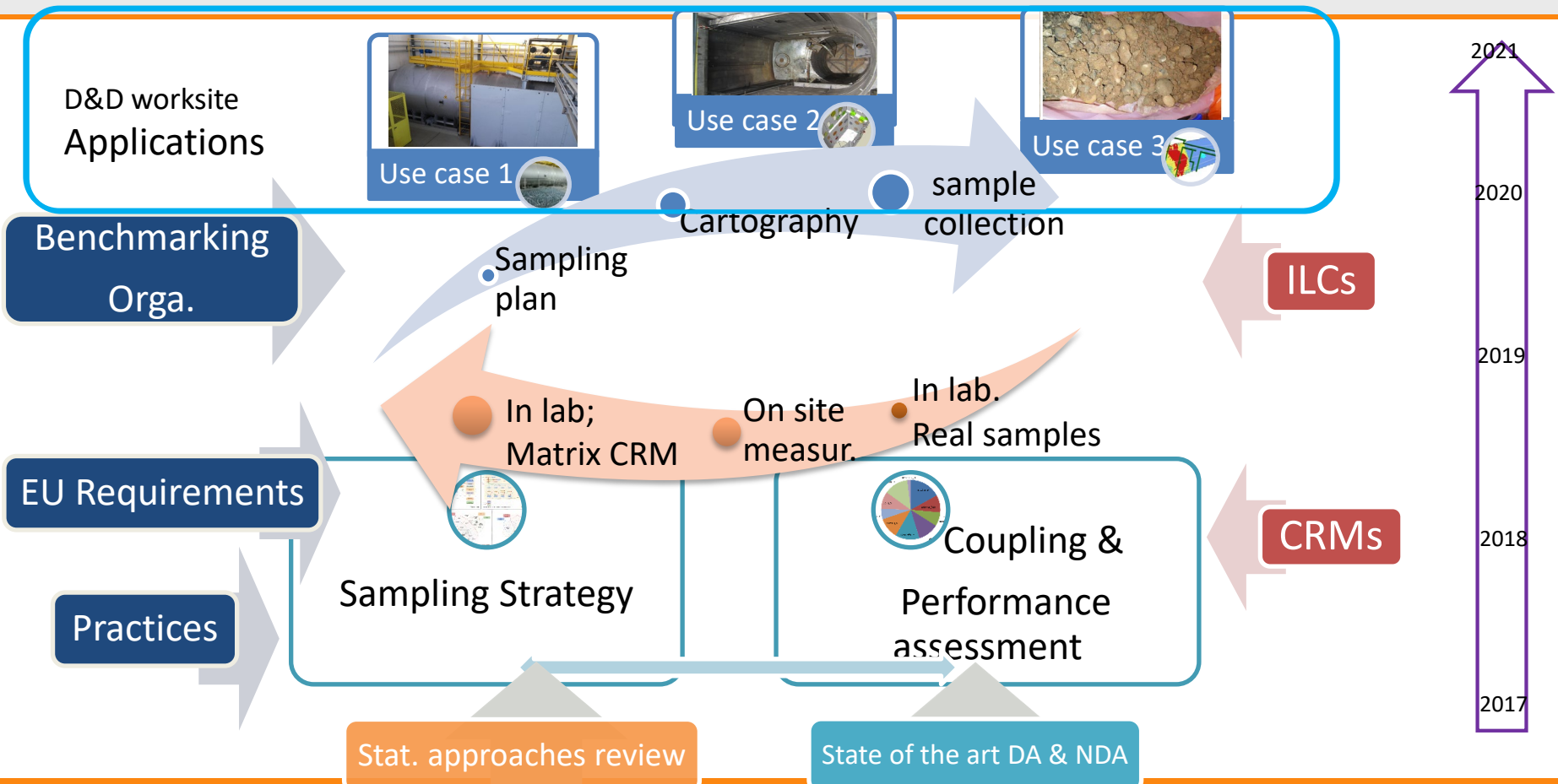
2

Decommissioning of a nuclear reactor - MoI (SCK/CEN)

3

Post accidental land remediation - (CEA)

Implementation: 3 main areas

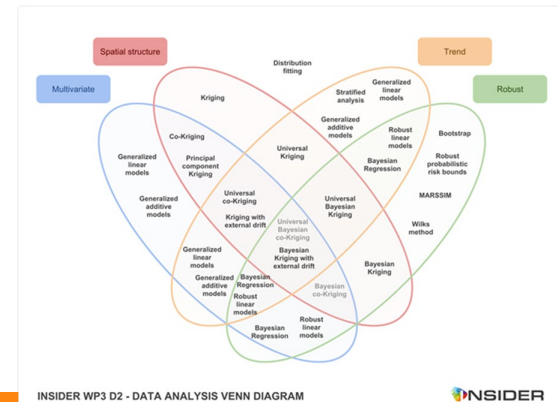
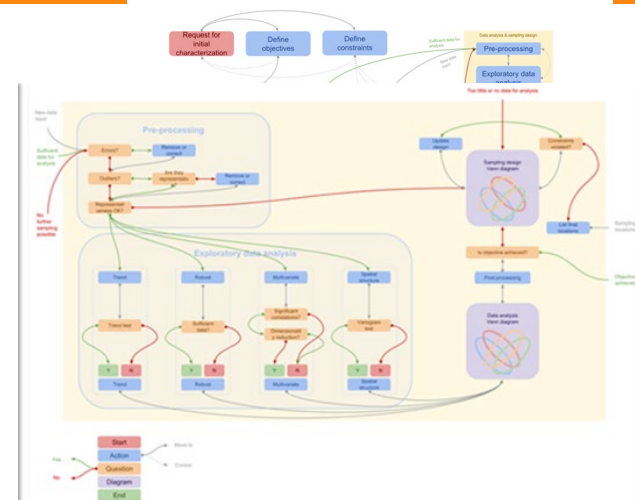


Global Statistical approach

- ❑ Support to sampling strategy and sampling design definition
- ❑ Waste-led approach
- ❑ Coupling sampling and characterization methods
 - ❑ gathering all possible data

Data analysis associated to sampling design

- ❑ Variables of interest and statistical indicators
- ❑ Data processing (pre and post analysis):
 - ❑ **Univariate or Multivariate** data analysis
 - ❑ Presence of **Spatial structure**
 - ❑ Presence of **Spatial trends**
 - ❑ Requirement for **Robust methods**



□ D&D Matrix Reference materials

□ Heavy concrete: *Ba-133, Co-60, Eu-152,154*

□ Homogenised doped real samples

□ Effluent solution:

□ Doped solution

□ Development of liquid-liquid micro-extraction

□ Microsystem-based analytical protocol for the extraction and purification of a radionuclide (^{55}Fe) prior to its analysis

□ **Microchannel** : 100 μm width; 40 μm depth; 8, 12, or 20 cm lengths

□ Ethyl acetate as the organic phase,

□ Cupferron in aqueous phase

□ two stage extraction

□ Measurements of Fe extraction yields :

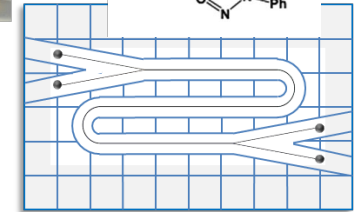
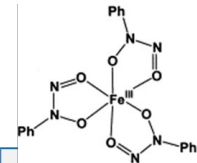
□ 45% in 1 sec in single-stage microsystem (protocol 1)

□ 60% in 1.35 sec in double-stage microsystem (protocol 2)

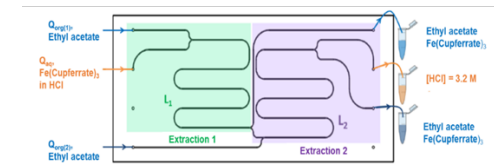
Main contaminants	Activity concentration range (Bq/g)
Ni-63	1-10
Sr-90	1-100
Pu-238	0.1-10
Pu-239	0.1-10
Am-241	1-10
Co-60	0.1-10
Cs-137	1-200
Fe-55	0.1-5
Pu-241	1-50
U-238	0.1-10



Complex formation



Single-stage extraction chip



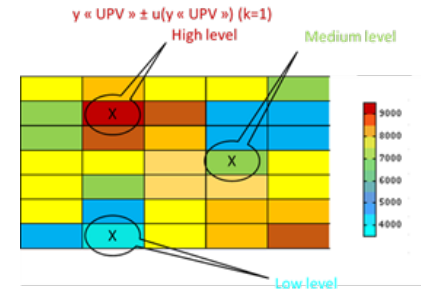
Two-stage extraction chip

S. RASSOU et al 2019 submitted

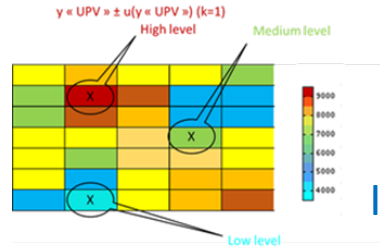
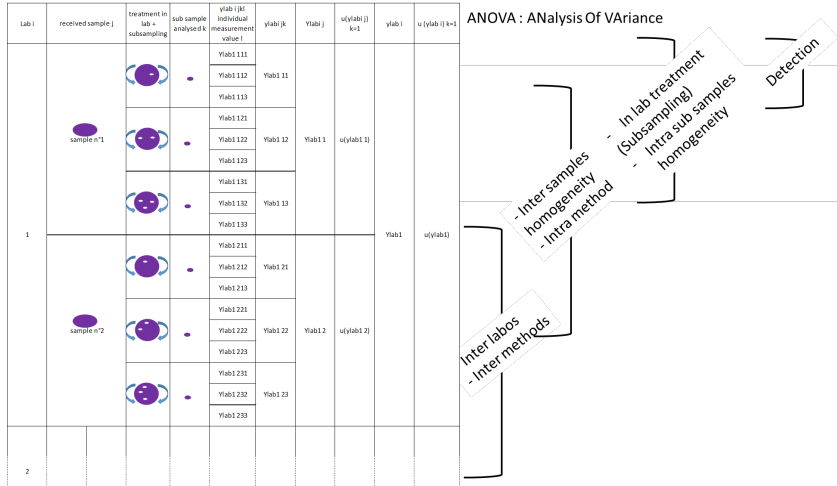
- ❑ Test **the ability of different techniques/methods** (proficiency test) to carry out measurements
- ❑ Estimate **the measurement (in lab or in situ) uncertainty** on synthetic and real samples
- ❑ Try to establish a **complete uncertainty budget** including every step of the INSIDER methodology (geostat & measurement)

❑ Interlaboratory comparisons organisation on

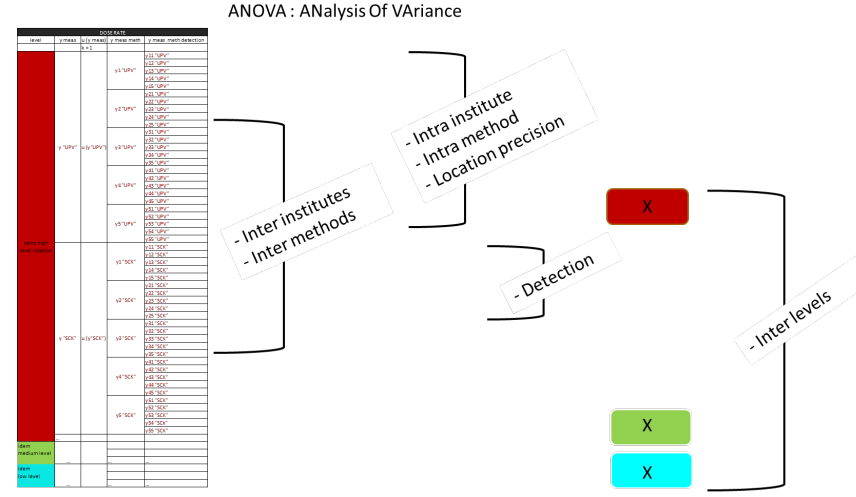
- ❑ Reference samples : proficiency test
 - ❑ In Lab DA and NDA
 - ❑ Reference materials produced within the project by WP4
- ❑ Real samples : benchmarking
 - ❑ Organize benchmark tests for in situ measurements(NDA)
 - ❑ in lab analysis (DA and NDA)
 - ❑ Homegenised real samples collected



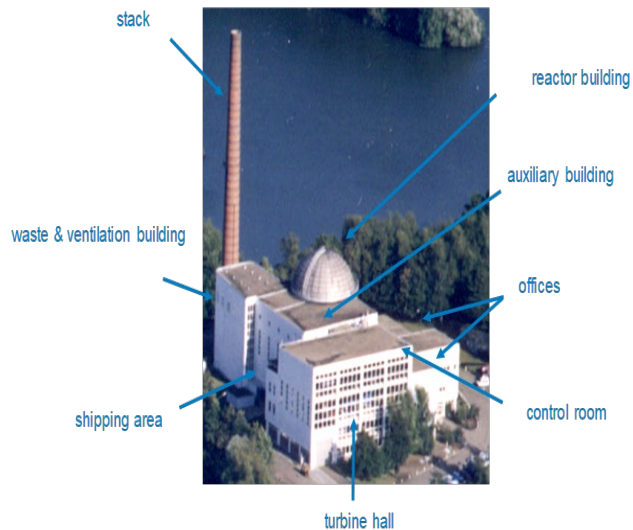
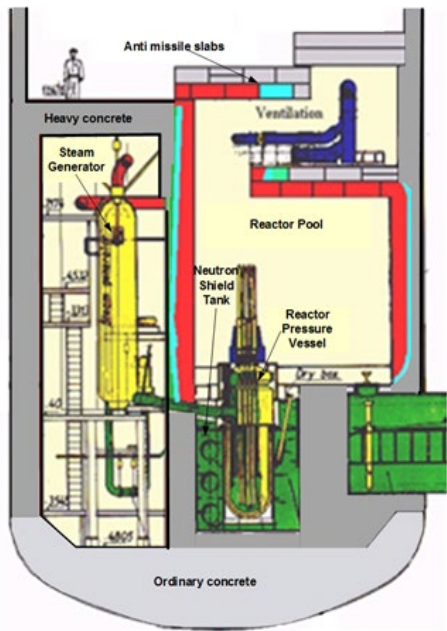
In Lab DA and NDA methods



In situ measurement techniques



BR 3 Reactor biological shield



In situ analysis performance assessment

in situ analysis :

- Sampling strategy: interest of small data set
- Improvement through performance assessment:

measurand = $(X \pm U)$ unit

Dose rate

Total gamma

^{133}Ba , ^{152}Eu , ^{154}Eu , ^{60}Co (γ spectro)

Validated analytical method:
Accuracy = Trueness + precision

Measurement process:

5 measurements with detector in the
fixed position

5 measurements with
removing/replacing detector

= 25 measurements

by detector to source distance

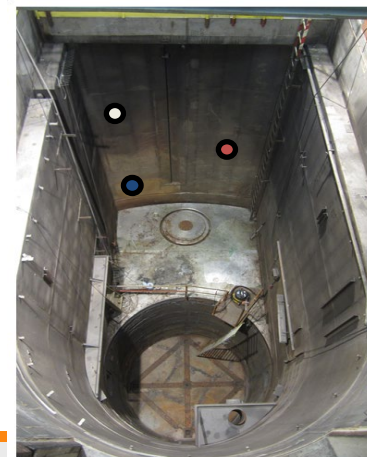
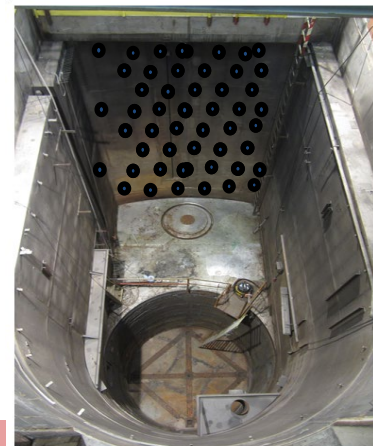
by measurement point

With 2 detector-to-source distances

With 3 measurement points

Detector
positioning
uncertainty

Detector
location
uncertainty
Activity or
concentration
level uncertainty



in situ inter-teams
comparisons :
5 different teams

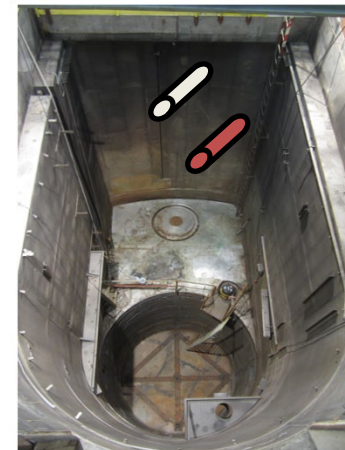
Interlaboratory comparison contribution

□ In lab. analysis :

- Sampling strategy: reduced number of samples
- measurand = $(X \pm U)$ unit

Using validated analytical method:
Accuracy = Trueness + precision

^{133}Ba , ^{152}Eu , ^{154}Eu ,
 ^{60}Co (gamma spectrometry)
total gamma
 ^3H , ^{14}C , ^{41}Ca



WP6



Interlaboratory
comparisons on
synthetic samples

WP6



Interlaboratory
comparison on
real samples

Method
performance



Concrete CRM

lab 1
lab 2



U hom



U hom

lab 1
lab 2



lab 1
lab 2



Measurand



Liquid effluent tank storage at JRC

Main RN to measure:

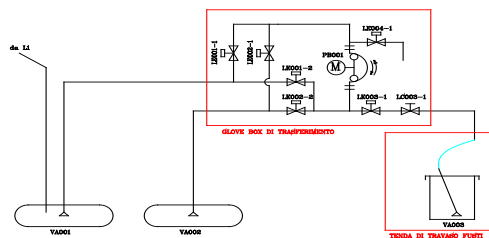
^{14}C , ^{41}Ca , ^{63}Ni , ^{79}Se , ^{90}Sr , ^{93}Zr , ^{99}Tc , ^{107}Pd , ^{147}Pm , ^{151}Sm e ^{241}Pu , ^{55}Fe ,
 ^{59}Ni , ^{93}Mo , ^{129}I , ^{60}Co , ^{94}Nb , ^{134}Cs , ^{137}Cs , ^{152}Eu , ^{154}Eu , ^{241}Am , ^{235}U , ^{238}U ,
 ^{237}Np , ^{238}Pu , $^{239+240}\text{Pu}$...



Which measurements?

- Dose rate
- Total gamma
- Gamma spectrometry

- after homogenisation with stirrers in operation
- after deposition of the solid fraction after long stop of the stirrers



In lab analysis on samples

INSIDER project perspectives

- ❑ **Innovative metrological study** based on a multidisciplinary network and **D&D key activities**
 - ❑ New D&D matrix reference materials development
 - ❑ Intercomparisons on real samples and Inter-team
 - ❑ **Analytical innovation** needs identification, development and implementation
 - ❑ Correlation and **scaling factors: Improvement of accuracy estimation of traces (DTM RN)**
 - ❑ **Advanced integrated approach for site radiological characterisation and automation of characterization process...**
 - ❑ Decommissioning operating **experience**

- ❑ **Methodological guides** updated according to benchmarking feedback
 - ❑ Established link with **standardisation commissions (ISO)** for future international standards
 - ❑ Contribution to **European learning (ELINDER)**
 - ❑ Interface with other **EU initiative (SHARE, METRODECOM projects)**

- ❑ Potential **further opening of the project**
 - ❑ Extension/application of the methodology and approaches : historic wastes, graphite reactors...
 - ❑ Interface with digital tools: Imaging, virtual and augmented reality

INSIDER

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Planning for tomorrow
 By 2050, more than half of today's 1,400 UK nuclear reactors are expected to have to be decommissioned. Smart applications and waste re-use will be key to ensure the amount of radioactive waste and nuclear potential is met.

A wide diversity
 Nuclear industry represents a wide variety of nations and experiences.

This project received funding from the European Research and Training Programme 2014-2018 under grant agreement 755554.

18 partners

What INSIDER will achieve

- Define the safety strategy, assess concrete conditions to be met when
- Assess the performance of available environmental technology (chemicals and tools) to establish an appropriate strategy and a realistic basis for decision making.
- Establish common methodologies to design, evaluate, generate safety assessment and financing options (SA) of nuclear power plants, and ensure the development of nuclear facilities under decommissioning.
- Apply the methodologies to real scenarios under decommissioning to evaluate their impact on the future.

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THANK YOU for your attention

Any questions?



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